# Formerly Utilized MED/AEC Sites Remedial Action Program 

# Radiological Survey of the Museum of Science and Industry, 57th Street and Lake Shore Drive, Chicago, Illinois 

February 1979
Final Report

Prepared for
U.S. Department of Energy

Assistant Secretary for Environment Division of Environmental Control Technology

## Formerly Utilized MED/AEC Sites Remedial Action Program

# Radiological Survey of the Museum of Science and Industry, 57th Street and Lake Shore Drive, Chicago, Illinois 

U.S. Department of Energy

Assistant Secretary for Environment
Division of Environmental Control Technology
Washington, D.C. 20545

## Available from:

# National Technical Information Service (NTIS) 

U.S. Department of Commerce

5285 Port Royal Road Springfield, Virginia 22161

Price: Printed Copy: \$5.75
Microfiche: \$3.00

## PREFACE

This is one of a series of reports resulting from a program initiated in 1974 by the Atomic Energy Commission (AEC) for determination of the condition of sites formerly utilized by the Manhattan Engineer District (MED) and the AEC for work involving the handling of radioactive materials. Since the early $1940^{\prime} \mathrm{s}$, the control of over 100 sites that were no longer required for nuclear programs has been returned to private industry or the public for unrestricted use. A search of MED and AEC records indicated that for some of these sites, documentation was insufficient to determine whether or not the decontamination work done at the time nuclear activities ceased is adequate by current guidelines.

This report contains the results of surveys of the current radiological condition of the Museum of Science and Industry, 57 th Street and Lake Shore Drive, Chicago, Illinois. Findings of this survey indicate there is no identifiable residual radioactivity remaining at this facility from operations conducted by the MED and AEC during the period 1946 thru 1953.

This survey was performed by the folzowing Heazth Physics personnel of the Occupational Health and Safety Division, Argonne National Laboratory, Argonne, Iľinois: R. A. Wynveen, W. H. Smilh, C. J. Mayes, P. C. Gray, D. W. Reilzy.

## CONTENTS

Introduction ..... 1
Survey Techniques ..... 2
General ..... 2
Instrumentation ..... 2-3
Smear Surveys ..... 3-4
Air Samples ..... 4
Soil Samples ..... 4-6
Analysis of Survey Results ..... 6
General ..... 6
Instrument Surveys ..... 7
Smear Surveys ..... 7
Air Samples ..... 7-8
Soil Samples ..... 8
Findings ..... 8
Table 1 - Data Sheets Showing Room Survey Results ..... 9-12
Figure 1 - Survey Locations ..... $13-18$
1 A - East Pavilion-Ground Floor ..... 13
1B - East Pavilion-First Floor. ..... 14
1C - East Pavilion-First Balcony ..... 15
1D - East Pavilion-Roof ..... 16
1E - Central Pavilion-Second Balcony. ..... 17
1F - Soil Sample. ..... 18
Table 2 - Instrumentation Used in Survey. ..... 19
Table 3 - Instrument Background Readings. ..... 20
Figure 2 - Gamma Spectrum Analysis of Floor Tile ..... 21
Table 4 - Radon Concentration Determinations. ..... 22
Figure 3 - Soil Sampling Procedure and Processing Diagram. ..... 23
Table 5 - Soil Sample Weights ..... 24
Table 6 - LFE Soil Analysis Procedure for Total Uranium and Gamma-Emitting Nuclides ..... 25
Table 7 - Ge(Li) Spectrum and Uranium Fluorometric Analyses Results ..... 26
Table 8 - Background Soil Sample Data ..... 27

## CONTENTS (cont'd.)

| Appendix | 1 - Conversion Factors . . . . . . . . . . . . . . . | $28-32$ |
| :--- | :--- | :--- | :--- |
| Appendix | 2 - Air Sample Data Sheets . . . . . . . . . . . . . | $33-42$ |
| Appendix | 3 - Normal Uranium Calculations . . . . . . . . . . | $43-44$ |

## INTRODUCTION

During the Manhattan Engineer District/Atomic Energy Commission (MED/AEC) Era, Argonne National Laboratory (ANL) occupied space at the Museum of Science and Industry, Chicago, Illinois. From August 15, 1946 until July 1, 1949, ANL occupied 36,000 square feet on the ground, first, and balacony floors of the East Pavilion. From August 15, 1946 until July 15, 1953, ANL also occupied 16,000 square feet in the 2nd Balcony of the West Court. The actual use of the facili,y is unknown. Although most of the area was believed to be occupied as office space, some handing of radioactive materials was known to have taken place. The type and activity of these materials is unknown.

Personnel involved with the facility during ANL's occupation recalled at least one spill of radioactive material near the service elevator on the ground floor of the East Pavilion and its subsquent decontamination.

Due to the uncertainty of the use of the facility, a radiation survey of the above area was undertaken from January 11, 1977 until Apri1 13, 1977. This survey was performed on an intermittent basis to minimize the disturbance of the Musuem's daily operations. The purpose of this survey was to determine if any detectable contamination remains as a result of the MED/AEC operation.

Part of the ground and main floors of the East Pavilion are presently occupied by the University of Chicago for storage and office space or are used for support of the Museum's operations. The 2nd Balcony of the West Court is now occupied as office space by the Museum of Science and Industry and the Academy of Interscience Methodology.

## SURVEY TECHNIQUES

## $2=$

ㅡㅡㄹㅡ́ssible original walls were surveyed to a height of seven feet and
 $\mathbb{E}=$ Eiled or painted．Even though these were not the original surfaces， $\because \equiv \equiv \equiv$ تere surveyed since the capability of detection was adequate to see $=\overline{=}=-$ the original structures underneath．A representative selective $\because=\equiv$ تrerheads such as pipes，vents and light fixtures was performed in areas $Z \equiv 二=-$－iginal structures were available．The roof of the East Pavilion was $=\equiv \equiv \approx=\therefore$ ．See Table 1 and Figure 1 for locations of accessible areas surveyed． ショニニニこ＝ごon
$\underline{\square}=$ Eypes of survey instruments were used（Table 2）．An Eberline FM－4G $\because \equiv \equiv$ EEection area of 325 square centimeters（ $\mathrm{cm}^{2}$ ），utilizing the Eberline $=-=-=E$ Ectronics，was used to survey the floors．A PAC－4G－3 with a hand－held $E=-=, E \mathrm{~cm}^{2}$ in area，was used to survey the walls and other accessible areas． $ニ ニ シ ミ=-$－izized mylar $\left[\sim 0.85\right.$ milligrams per square centimeter $\left.\left(\mathrm{mg} / \mathrm{cm}^{2}\right)\right]$ $\because ニ ー \square ミ ニ ミ$ used in both detectors．This allows for low energy detection and



 $\equiv ミ ニ ミ ー シ t c h e d ~ t o ~ t h e ~ a l p h a ~ m o d e ~ a n d ~ a ~ r e a d i n g ~ o f ~ t h e ~ a l p h a ~ a c t i v i t y ~ w a s ~$


三二 $\Xi=$－indow Geiger－Muieller（G－M）Detector，Eberline Model E－500B with a ニニニミ～－ミinch diameter window held three feet above the floor，was used to deter－ ニーシ $\because=-\therefore=$ iad an elevated count rate，a contact reading was obtained．

The End Window G-M Detector was calibrated using the gamma emissions from a Radium-226 ( ${ }^{226} \mathrm{Ra}$ ) calibration source. The PAC-4G-3 instruments were calibrated in the alpha mode using a flat plate infinitely thin Plutonium- 239 ( ${ }^{239} \mathrm{Pu}$ ) standard and in the beta mode with a flat plate infinitely thin Strontium-90-Yttrium-90 $\left({ }^{90} \mathrm{Sr}-{ }^{90} \mathrm{Y}\right)$ standard. The instruments were calibrated to an apparent 50\% geometry.

It must be realized that the numerous isotopes that could be encountered will exhibit emission energies differing from that of ${ }^{239} \mathrm{Pu}$ and ${ }^{90} \mathrm{Sr}-{ }^{90} \mathrm{Y}$ utilized in the calibration. When detecting known isotopes that emit alpha and beta energies differing from that of the standards, a conversion factor is developed to determine the appropriate yield.

## Smear Surveys

Smears were taken throughout the East Pavilion and West Balcony areas of the Museum. Only original structures and components such as walls, floors, pipes and vents were smeared. All smears were taken with No. 1 Whatman filter paper, 4.25 centimeters (cm) in diameter. Smears of one square foot were normally taken. If an area was found which had a higher than normal background, a smear of $100 \mathrm{~cm}^{2}$ was taken. A smear of $100 \mathrm{~cm}^{2}$ was also taken if an area indicated excessive dirt loading. The smears were counted in groups of ten using the 10-Wire Flat Plate Gas Proportional Detector, developed at ANL, utilizing an Eberline Mini Scaler Model MS-2. One smear of each group was removed and counted in a Nuclear Measurement Corporation Proportional Counter - $3 \mathrm{~A}(\mathrm{PC}-3 \mathrm{~A}) 2 \pi$ Internal Gas Flow Counter using a mylar spun top. This procedure was used as an additional means of checking the smear samples. In addition, any smears indicating elevated amounts in the lo-Wire Assembly, were also counted in the more sensitive PC-3A counter. Smears were counted in both detectors for alpha and beta activity. Appendix 1 includes the instrumentation and smear count conversion factors used.

Table 1 includes the room survey readings while the maps in Figure 1 indicate the location of the smears. A number, $n$, indicates the location of that smear in the room. A number, (n), indicates a smear of an overhead structure. A number $n$, indicates an elevated direct reading.

## Air Samples

Air samples were collected using a Filter Queen air sampling device. The air samples were taken at a flow rate of 15 cubic meters per hour ( $M^{3} / \mathrm{hr}$ ) on a $200 \mathrm{~cm}^{2}$ sheet of Hollingsworth-Vose (HV-70-9 mil) filter media which collected the particulates present in the air. A $10 \%$ portion, 5 cm in diameter, was removed from the filter media and counted in the NMC PC-3A $2 \pi$ Internal Gas Flow Counter, utilizing a mylar spun top for both alpha and beta activity. Sampling results were used to determine radon concentrations and the presence of any long-lived activity. Air sample data is presented in Appendix 2.

## Soil Samples

In addition to the survey inside the building, soil corings were taken at selected locations outside the East Pavilion of the Museum to determine the deposition, if any, of isotopes that could have been spilled or released from the East Pavilion. Radiochemical (fluorometric) and gamma spectrum analysis were conducted on these soil samples.

The corings were effected using a four (4) inch in diameter by six (6) inch in length right circular cylinder; commonly called a hole cutter. This device is normally used for cutting holes for the cups in golf courses.

Each core was 1 foot in length and divided into four (4) segments. Starting from the surface, three (3) separate two (2) inch segments are cut, bagged, and marked $A, B$ and $C$ respectively; the final segment a six (6) inch section was marked D.

The reason for the segmented coring is to determine what, if any, contaminant migration has occurred, to reduce the dilution of lower level soil with the upper level segments in respect to the surface deposition of the contaminants or vice versa, and to reveal any overburden or back fill that may have occurred over the years.

Three soil samples were taken from the grounds adjacent to the East Pavilion of the Museum. Figure $1 F$ indicates the soil sample locations.

Background data for the soil sample analysis (Table 8 ) were obtained from a number of soil samples taken from the Chicago area. This information was obtained from the Environmental Monitoring Section of the Occupational Health and Safety (OHS) Division of ANL.

All soil samples were processed at ANL (Figure 3) and shipped to a commercial laboratory (LFE Environmental Analysis Laboratories) for radiochemical (fluorometric) and gamma spectrum analysis. Their soil analysis procedure is described in Table 6,

Sample preparation consisted of weighing the samples in their entirety and then drying for approximately 24 hours at $80^{\circ}$ Centigrade. All samples were then reweighed, put into mill jars ( 2.3 gallon) and milled until a sufficient amount of the soil sample would pass a No. 30 standard sieve. At no point were the rocks and heavy material ground or pulverized since this material would act as a diluent and hence lower the concentration per unit volume of deposited material.

After sufficient milling, the material was sieved using a No. 30,600 micron ( $\mu$ ) standard stainless sieve. The rocks and dross vs. sieved material (< $600 \mu$ ) was segregated, bagged, and weighed separately. Soil sample weights are given in Table 5.

Aliquots of the sieved material were then loaded into screw top plastic containers. The amount varied according to the type of analysis to be performed; 100 grams for gamma and radiochemical (fluorometric) analysis and 10 grams for radiochemical (fluorometric) only. Every effort was made throughout the sample preparation operations to reduce or eliminate cross contamination. Soil samples which were suspected of containing elevated amounts of radioactivity were processed in equipment separate from the soil samples considered to contain background levels. All items of equipment were scrubbed and air dried prior to the introduction of the next sample.

## ANALYSIS OF SURVEY RESULTS

## General

All data, including diagrams of survey locations, are attached to this report. This section discusses the results of the survey and the findings therein. Instrument readings and smear results were normalized to units of disintegrations per minute per one hundred square centimeters ( $\mathrm{dpm} / 100 \mathrm{~cm}^{2}$ ). (See Appendix 1 for the conversion factors used.) All data is reported in net counts, i.e., the background counts have been subtracted from the gross counts prior to converting from counts per minute per one hundred square centimeters ( $\mathrm{cpm} / 100 \mathrm{~cm}^{2}$ ) to $\mathrm{dpm} / 100 \mathrm{~cm}^{2}$. The beta mode readings are compensated for any alpha contribution. The room background levels varied somewhat due to the construction materials in them. Table 3 provides an average background reading for all modes of the different instruments used.

The areas accessible for survey varied from room to room. Areas accessible for survey are presented in Table 1. The average percent of the total accessible areas was $50 \%$ for the floors and $40 \%$ for the walls.

## Instrument Surveys

All indicated areas were surveyed and no radioactivity above background levels could be detected except in the following four rooms.

Room C-340 - This room is a small instrument shop where a marked Cobalt-60 $\left({ }^{60} \mathrm{Co}\right)$ source was found in the cabinet. A direct reading with an End Window G-M Detector was 80 milliRoentgens per hour ( $\mathrm{mR} / \mathrm{hr}$ ) at contact. When the detector was held three feet away from the source in its shielded container, no radiation above background levels, < $0.03 \mathrm{mR} / \mathrm{hr}$, could be detected.

Room E-201 and Restrooms on 2nd Balcony - These washrooms contained a white tile on the floors. This tile was also noted in other restrooms of the Museum. These tiles indicated $8.1 \times 10^{3} \mathrm{dpm} / 100 \mathrm{~cm}^{2}$ Potassium- $40(40 \mathrm{~K})$ with the PAC-4G-3 in the beta mode. No alpha activity was detected. No radiation above background levels could be found from the tile using the End Window G-M Detector. No activity was detected from floor tile smears. It was determined from a gamma spectral analysis that the tile contained elevated amounts of ( ${ }^{40} \mathrm{~K}$ ) which would cause an elevated reading. (See Figure 2 for gama emission spectra.) All general background readings taken at three feet above the floor level were less than $0.03 \mathrm{mR} / \mathrm{hr}$.

## Smear Surveys

No contamination above background levels was detected on any smears.

## Air Samples

The air sampling results are presented in Table 4. The variation of the data results do not appear to be a result of any MED/AEC operation, but rather the variation reflects the differences in the construction materials used throughout the facility. Other factors such as the ventilation of the room can cause the radon concentrations to vary. All radon concentrations determined are below the maximum permisible concentrations (MPC) for an uncontrolled area às listed in the "Standards for Protection Against Radiation," Code of Federal Regulations, Title 10, Part 20,
typendix B (April 30, 1975), (10CFR20). These concentrations are found to be within Vie normal expected levels of radon.

三=11 Samples
Results submitted by LFE Environmental Analysis Laboratories, as listed in Záole 7, are reported in picocuries. per gram ( $\mathrm{pCi} / \mathrm{g}$ ) for the Germanium (Lithium) $[\mathrm{f} \in(\mathrm{L} 1)]$ spectral analysis and in micrograms per gram ( $\mu \mathrm{g} / \mathrm{g}$ ) for the uranium Eluorometric analysis. The latter concentrations were converted to $\mathrm{pCi} / \mathrm{g}$ by means of the example calculation as shown in Appendix 3.

The background data is presented in Table 8. The background samples indicate تormal uranium concentrations ranging from 0.03 to $2.0 \mathrm{pCi} / \mathrm{g}$. Results of soil samples taken at the Museum of Science and Industry indicate a gerneral normal uranium Dackground concentration in the soil.

## FINDINGS

The survey results show that no radioactive contamination above background was detected throughout the areas used for MED/AEC activities. However, a small ${ }^{60}$ Co source which was used as a static eliminator was found in Room C-340. The floor tiles which were used in the restrooms, showed elevated levels of naturally occurring ${ }^{40} \mathrm{~K}$. Neither of these are a result of any MED/AEC operations. The results of the soil sample analysis shows no elevated readings above the natural background levels present in the soil from this region.

TABLE I
TATA SHOWING ROOM SURVEY RESULTS

| Room or Area No. | Percent <br> Accessib <br> Survey <br> Floor | of Area <br> le for <br> Wall | $\begin{aligned} & \text { Air } \\ & \text { Sample } \\ & \text { (pCi/l) } \end{aligned}$ | Beta <br> Floors | $\begin{aligned} & \text { Mode }{ }^{(1)} \\ & (\mathrm{dpm} / 1 \\ & \text { Walls } \\ & \hline \end{aligned}$ | Direct Rea $00 \mathrm{~cm}^{2}$ ) <br> Overhead | ings <br> Other | Alpha <br> Floors | Mode Di (dpm) Walls | rect Readi $100 \mathrm{~cm}^{2}$ ) Overhead | gs <br> Other | $\begin{gathered} \text { End Wi } \\ \text { (mR } \\ \text { Contact } \end{gathered}$ | dow <br> hr) <br> 3 feet | Smear <br> Results $\left(\mathrm{dpm} / 100 \mathrm{~cm}^{2}\right)$ | Comments |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| E-1 | 70 | 20 | NS ${ }^{(2)}$ | $\mathrm{BKGD}^{(3)}$ | BKGD | .0Su ${ }^{(4)}$ | BKGD | $N A^{(6)}$ | NA | NA | NA | N(17) | BKGD | BKGD |  |  |
| E-2 | 80 | 25 | 1.25 | BKGD | BKGD | OSU | $\mathrm{NE}^{(5)}$ | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-3 | 70 | 20 | NS | BKGD | BKGD | OSU | BKGD | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-4 | 60 | 25 | NS | BKGD | 3KGD | OSU | BKGD | NA | NA | NA | NA | NN | EKGD | BKGD |  |  |
| E-4A | 40 | 25 | NS | BKGD | BKGD | OSU | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-4AA | 40 | 30 | NS | BKGD | BKGD | OSU | BKGD | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-7 | 95 | 30 | NS | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-8 | 40 | 40 | NS | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-9 | 50 | 50 | NS | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA. | NN | BKGD | BKGD |  | 1 |
| E-11 | 50 | 50 | NS | BKGD | BKGD | KBGD | NE | NA | NA | NA | NA | NN | BKGD | 3KGD |  |  |
| E-12 | 40 | 40 | NS | BKGD | BKGD | OSU | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-14 | 6 | 50 | 0.48 | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-15 | 20 | 1 C | 0.48 | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-16 | 20 | 20 | NS | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-17 | 50 | 50 | NS | BKGD | BKGD | BKGD | BKGD | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-18 | 30 | 20 | 1.5 | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |
| E-19 | 20 | 20 | NS | BKGD | BKGD | BKGD | NE | NA | NA | NA | NA | NN | BKGD | BKGD |  |  |

${ }^{(1)}$ Beta Mode detects both electromagnetic and particulate radiation.
${ }^{(2)}$ NS - (Not Selected) Air sample locations were chosen on a selected basis throughout the areas surveyed.
(3) BKGD (Background) Instrument Background Readings

| Floor Monitor | Beta Mode | Alpha Mode |
| :--- | :---: | :--- |
| PAC-4G-3 | $1500-2000 \mathrm{cpm} / 325 \mathrm{~cm}^{2}$ | $0-50 \mathrm{cpm} / 325 \mathrm{~cm}^{2}$ |
| PC-3A | $150-200 \mathrm{cpm} / 61 \mathrm{~cm}^{2}$ | $0-50 \mathrm{cpm} / 61 \mathrm{~cm}^{2}$ |
| 10 Wire | 50 cpm | 0.4 cpm |
|  | 500 cpm | 10 cpm |

(4) OSU (Overhead Stucture Unavailable) Floor and wall survey indicate no necessity to demolish existing structures to reach original overhead surfaces
(5) NE (Non-Existant) This location did not contain structural items calssified as "other" such as the following: ducts, louvers, pipes and vents.
(6) NA (Not Applicable) No activity detected above background in the beta mode; therefore, no alph mode survey was necessary.
(7) $N N$ Not Necessary) No activity was detected; therefore, no contact G-M End Window Survey was necessary.
data sheets showing room survey results


TABLE I


G-M End Window Detector read $<0.03 \mathrm{mR} / \mathrm{hr}$ at 3 feet above floor.
table I
data sheets showing room survey results


FIGURE 1A
SURVEY LOCATICNS OF EAST PAVILION - GROUND FLOOR


FIGURE 1B
SURVEY LOCATIONS OF EAST PAVILION - FIRST FLOOR


FIGURE 1C
SURVEY LOCATIONS OF EAST PAVILION - FIRST BALCONY


FIGURE 1D
SURVEY LOCATIONS OF EAST PAVILION - ROOF



FIGURE 1F



TABLE 2

INSTRUMENTATION USED IN SURVEY

| Type | Inventory Number | Probe Area | Window |
| :---: | :---: | :---: | :---: |
| Eberline Floor Monitor FM-4G utilizing a PAC-4G-3 | 181501 | $325 \mathrm{~cm}^{2}$ | $0.85 \mathrm{mg} / \mathrm{cm}^{2}$ |
| Eberline Floor Monitor FM-4G utilizing a PAC-4G-3 | 181581 | $325 \mathrm{~cm}^{2}$ | " |
| PAC-4G-3 | 165251 | $61 \mathrm{~cm}^{2}$ | " |
| " | 165252 | " | " |
| " | 165255 | " | " |
| " | 165256 | " | " |
| " | 183413 | " | " |
| " | 183414 | " | " |
| Eber1ine HP-90 Beta-Gamma End Window | 159006 | - | $1.4-2 \mathrm{mg} / \mathrm{cm}^{2}$ |
| Nuclear Measurement Corporation PC-3A-2m Internal Gas Flow Counter | 114969 | - | $0.85 \mathrm{mg} / \mathrm{cm}^{2}$ |
| Argonne National Laboratory Filter Queen Air Sampler using HV-70 filter media | - | - | - |
| Argonne National Laboratory <br> 10 Wire Flat Plate Gas <br> Proportioal Detector with <br> Eberline Mini Scaler MS-2 | 184343 | - | $0.85 \mathrm{mg} / \mathrm{cm}^{2}$ |

TABLE 3

## INSTRUMENT BACKGROUND READINGS


*Background readings were initially taken in the mobile laboratory and rechecked throughout the various areas inside the Museum of Science and Industry while surveying.

FIGURE 2
GAMMA SPECTRUM ANALYSIS OF FLOOR TILE


## TABLE 4

## RADON CONCENTRATION DETERMINATIONS

| Location | dpm/M ${ }^{3}$ | pCi/1 | \% of MPC* |
| :---: | :---: | :---: | :---: |
| E-2 | 2744 | 1.25 | 42 |
| E-14 | 1057 | 0.48 | 16 |
| E-15 | 1057 | 0.48 | 16 |
| E-18 | 3372 | 1.5 | 51 |
| E-114 | 1075 | 0.49 | 16 |
| E-117 | 1567 | 0.71 | 24 |
| E-201 | 2277 | 1.04 | 35 |
| E-202 | 1476 | 0.67 | 22 |
| South Hall (2nd Balcony West Court) | 683 | 0.31 | 10 |
| North Hall (2nd Balcony West Court) | 671 | 0.31 | 10 |

*The 10CFR20 MPC for Radon-222 ( ${ }^{222} \mathrm{Rn}$ ) in an uncontrolled area is $3 \times 10^{-9} \mu \mathrm{Ci} / \mathrm{cc}$ which equals $3 \mathrm{pCi} / 1$.

Example Calculation Room E-15

$$
1057 \mathrm{dpm} / \mathrm{M}^{3} \times \frac{1 \mathrm{pCi}}{2.22 \mathrm{dpm}} \times \frac{\mathrm{M}^{3}}{10^{3} 1}=0.48 \mathrm{pCi} / 1
$$



## TABLE 5

SOIL SAMPLE WEIGHTS

| Sample No. | Net Weight (grams) | Dry Weight (grams) | Sieved Weight (grams) | Rocks and Dross Weight (grams) |
| :---: | :---: | :---: | :---: | :---: |
| EP-1A | 887.8 | 688.9 | 666.0 | 145.0 |
| EP-1B | 749.0 | 582.0 | 551.6 | 3.5 |
| EP-1C | 740.8 | 579.6 | 557.2 | 13.5 |
| EP-1D | 1642.2 | 1281.8 | 1161.8 | 94.3 |
| EP-2A | 616.1 | 435.2 | 376.0 | 55.3 |
| EP-2B | 764.8 | 593.5 | 541.1 | 44.7 |
| EP-2C | 1050.0 | 833.0 | 766.4 | 60.0 |
| EP-2D | 2375.3 | 1944.5 | 1750.0 | 189.7 |
| EP-3A | 677.8 | 495.9 | 417.1 | 72.2 |
| EP-3B | 907.2 | 717.2 | 649.3 | 65.3 |
| EP-3C | 962.3 | 785.5 | 750.4 | 26.9 |
| EP-3D | 2142.2 | 1800.5 | 1563.3 | 222.9 |

## TABLE 6

LFE SOIL ANALYSIS PROCEDURE FOR TOTAL URANIUM AND GAMMA-EMITTING NUCLIDES

## Summary of Methods

A 60 milliliter ( ml ) volume of the received soil was counted in a petri dish for 500 minutes on a Ge(Li) detector over the energy range $0-1.5 \mathrm{MeV}$. This corresponded to between 60 to 100 g of soil, depending upon bulk soil density. Positive photopeaks above instrument background were converted to dpm using a line efficiency curve based upon a National Bureau of Standards Multi Gamma standard. The natural Thorium-232 ( $\left.{ }^{232} \mathrm{Th}\right)$ and ${ }^{226}$ Ra decay chains were calculated using the 0.910 MeV Actinium-228 $\left({ }^{228} \mathrm{Ac}\right)$ and 0.609 MeV Bismuth- $214\left({ }^{214} \mathrm{Bi}\right)$ photopeaks respectively. Cesium-137 is reported for each sample as a representative gamma emitter. Potassium-40 ( ${ }^{40} \mathrm{~K}$ ) was observed on all soil samples, as expected, but was not calculated or reported.

One gram of the soil sample was ashed and dissolved in $\mathrm{HF}-\mathrm{HNO}_{3}$ for the total uranium analysis. A 100- $\lambda$ aliquot of the dissolved sample was fused with $98 \%$ $\mathrm{NaF}-2 \% \mathrm{LiF}$ and the fluorescence determined using a Jarrell-Ash fluorometer. A quenching factor was determined for each sample by using an internal spike.

TABLE 7

Ge(Li) SPECTRUM AND URANIUM FLUOROMETRIC ANALYSES RESULTS

| Sample No. | $\mathrm{Ge}(\mathrm{Li})$ Spectra $\mathrm{pCi} / \mathrm{g}$ received wt $\pm \sigma$ (1) |  |  | Uranium |  |
| :---: | :---: | :---: | :---: | :---: | :---: |
|  | ${ }^{137} \mathrm{Cs}$ | ${ }^{232}$ Th Decay Chain | ${ }^{226}$ Ra Decay Chain | $\mu \mathrm{g} / \mathrm{g} \pm \sigma^{\mathrm{Ur}}$ | Lum $\mathrm{pCi} / \mathrm{g} \pm \sigma^{(3)}$ |
| EP-1A | $1.43 \pm 0.07$ | $0.8 \pm 0.2$ | $0.75 \pm 0.08$ | $3.5 \pm 0.4$ | $2.4 \pm 0.3$ |
| EP-1B |  |  |  | $2.6 \pm 0.4$ | $1.8 \pm 0.3$ |
| EP-1C |  |  |  | $1.3 \pm 0.4$ | $0.9 \pm 0.3$ |
| EP-1D |  |  |  | $3.1 \pm 0.4$ | $2.2 \pm 0.3$ |
| EP-2A | $0.98 \pm 0.05$ | $0.9 \pm 0.1$ | $0.83 \pm 0.07$ | $2.2 \pm 0.4$ | $1.5 \pm 0.3$ |
| EP-2B |  |  |  | $2.2 \pm 0.5$ | $1.5 \pm 0.3$ |
| EP-2C |  |  |  | $1.9 \pm 0.6$ | $1.3 \pm 0.4$ |
| EP-2D |  |  |  | $1.9 \pm 0.4$ | $1.3 \pm 0.3$ |
| EP-3A | $1.05 \pm 0.06$ | $0.6 \pm 0.2$ | $0.93 \pm 0.09$ | $2.6 \pm 0.4$ | $1.8 \pm 0.3$ |
| EP-3B |  |  |  | $3.5 \pm 0.5$ | $2.4 \pm 0.3$ |
| EP-3C |  |  |  | $4.1 \pm 0.5$ | $2.9 \pm 0.3$ |
| EP-3D |  |  |  | $2.4 \pm 0.4$ | $1.7 \pm 0.3$ |
| LFE Blank | $0 \pm 0.06$ | $0 \pm 0.1$ | $0 \pm 0.06$ | $0 \pm 0.2$ | $0 \pm 0.1$ |

(1) One standard deviation due to counting statistics.
(2) Data Results from LFE.
(3) ANL Conversion from Appendix 3.

TABLE

## background soil sample data*

## Cesium-137, Thorium, and Uranium in Soil 1976 concentrations in $\mathrm{pCi} / \mathrm{g}$

| Date <br> Collected | Location | Cesium-137 | Thorium-232 | Uranium |
| :--- | :--- | :--- | :--- | :--- |
| July 22 | Argonne Area | $0.3 \pm 0.1$ | $0.21 \pm 0.04$ | $1.3 \pm 0.1$ |
| July 22 | Argonne Area | $0.1 \pm 0.1$ | $0.49 \pm 0.04$ | $2.0 \pm 0.1$ |
| July 22 | Argonne Area | $0.3 \pm 0.1$ | $0.48 \pm 0.04$ | $1.5 \pm 0.1$ |
| October 18 | Argonne Area | $0.1 \pm 0.1$ | $0.65 \pm 0.07$ | $1.5 \pm 0.1$ |
| October 18 | Argonne Area | $0.3 \pm 0.1$ | $0.43 \pm 0.04$ | $1.4 \pm 0.1$ |
| October 18 | Argonne Area | $0.4 \pm 0.1$ | $0.39 \pm 0.04$ | $1.3 \pm 0.1$ |

## Off-Site

June 22
McKinley Woods
State Park, IL
$0.4 \pm 0.1$
$0.16 \pm 0.02$
$0.9 \pm 0.1$

McCormick Woods
$0.3 \pm 0.1$
$0.22 \pm 0.02$
$1.2 \pm 0.1$
Brookfield, IL
Bemis Woods
$0.4 \pm 0.1$
$0.18 \pm 0.01$
$1.6 \pm 0.1$
June 23
Hinsdale, IL
St. Joseph, MI
$0.4 \pm 0.1$
$0.20 \pm 0.02$
$0.3 \pm 0.1$
October 12
Willow Springs, IL $0.5 \pm 0.2$

- $\quad 1.0 \pm 0.1$

Dresden Lock \&
$0.4 \pm 0.1$
$0.45 \pm 0.03$
$1.6 \pm 0.1$ Dam, IL

Average
$0.4 \pm 0.1$
$0.24 \pm 0.14$
$1.1 \pm 0.5$
*These results are transcribed from "Environmental Monitoring at Argonne National Laboratory Annual Report for 1976" (ANL-77-13) by N. W. Golchert, T. L. Duffy and J. Sedlet. These measurements are presented in Table 13, on page 47 of the report.

## APPENDIX I

## CONVERSION FACTORS

INSTRUMENTATION

Below are the conversion factors used to obtain the readings in units of disintegrations per minute per $100 \mathrm{~cm}^{2}\left(\mathrm{dpm} / 100 \mathrm{~cm}^{2}\right)$.

I Conversion Factors

Floor Monitor (FM-4G)
PAC-4G-3

| To $100 \mathrm{~cm}^{2}$ | 0.31 | 1.6 |
| :--- | :--- | :---: |
| cpm to dpm (alpha) | 2 | 2 |
| cpm to dpm (beta) | 2 | 2 |
| cpm to $\mathrm{dpm}\left(\mathrm{K}^{40}\right)$ | - | 16.5 |

II Derivation of Conversion Factors
Floor Monitor (FM-4G)
Window Area: $\quad 325 \mathrm{~cm}^{2}$
Conversion to $100 \mathrm{~cm}^{2}=.31$ times floor monitoring reading

PAC-4G-3
Window Area: $\quad-61 \mathrm{~cm}^{2}$
Conversion to $100 \mathrm{~cm}^{2}=1.6$ times PAC reading
$2 \pi$ Internal Gas Flow Counter, PC-3A
Geometry: Mylar Spun Top - 0.43
Mylar Spun Top Counting (window double aluminized mylar ~0.85 $\mathrm{mg} / \mathrm{cm}^{2}$ ) utilizes the well of the PC-3A and is a method developed and used by the Argonne National Laboratory Health Physics Section for negating the dielectric effect in counting samples on non-conducting media.

## APPENDIX I (cont'd.)

SMEAR COUNT

The conversion factors for $\mathrm{cpm} / 100 \mathrm{~cm}^{2}$ to $\mathrm{dpm} / 100 \mathrm{~cm}^{2}$ are given below.

I CONVERSION EQUATION (ALPHA)

## cpm-Bkgd

$g \times$ bf $\times$ sa $\times$ waf $=d p m A l p h a$
A geometry (g) of 0.43 is standard for all flat plate counting.
A backscatter factor (bf) of 1.0 is used when determining alpha activity on a filter media.

The self-absorption (sa) was assumed to be 1 unless otherwise determined.
If the energies of the isotope were known, the appropriate window air factor (waf) was used; if the energies of the isotopes were unknown the waf of ${ }^{239} \mathrm{Pu}$, which is . 713, was used.

II CONVERSION EQUATION (BETA)
$\frac{\mathrm{cpm}-\text { (Beta Bkgd }+ \text { Alpha } \mathrm{cpm})}{\mathrm{g} \times \mathrm{bf} \times \mathrm{sa} \times \mathrm{waf}}=$ dpm Beta
A geometry (g) of 0.43 is standard for all flat plate counting.
A backscatter factor (bf) of 1.1 is used when determining beta activity on a filter media.

The self-absorption (sa) was assumed to be 1 unless otherwise determined.
If the energies of the isotopes were known, the appropriate window air factor (waf) was used; if the energies of the isotopes were unknown, the waf of ${ }^{90} \mathrm{Sr}-{ }^{90} \mathrm{Y}$, which is 0.85 was used.

APPENDIX I (cont'd.)

## RADON DETERMINATION

This attachment summarizes the air sampling calculations for samples collected using Argonne National Laboratory designed air sampler with HV-70 filter media. The attachment includes the basic assumptions and calculations used to derive the air concentrations.

## I. Radon Concentrations Based on RaC' Results

The following postulates are assumed in deriving the Radon-222 ( ${ }^{222} \mathrm{Rn}$ ) concentrations as based on the $\mathrm{RaC}^{\prime}$ alpha count results.

1. RaA, RaB, RaC, RaC', are in equilibrium.
2. RaA is evident only in the first count and not the 100 minute decay count.
3. That one-half of the Radon progeny is not adhered to airborne particulate, and therefore, not evident on the filter media.
4. The geometry factor (g) is 0.43 for both the alpha and beta activity.
5. The backscatter factor (bf) of 1.0 is used for the alpha activity which is determined from RaC'.
6. The sample absorption factor (sa) for RaC' is 0.77.
7. The window air factor (waf) for $R a C$ ' is 0.8 .
8. RaB and RaC being beta emitters, are not counted in the alpha mode.
9. The half-life of the Radon progeny is approximately 36 minutes, based on the combined $R a B$ and $R a C$ half-lives.
10. No long-lived alpha emitters present as evidenced by the final recount.
11. For all practical purposes, $\operatorname{RaC}^{\prime}$ decays at the rate of the composite of RaB and RaC which is approximately 36 minutes.

## APPENDIX I (cont'd.)

## RADON DETERMINATION (cont'd.)

II. Equations Used to Derive Air Concentrations

$$
\begin{aligned}
& N_{o}=\frac{N}{-\lambda t} \\
& \text { e } \\
& \text { Where: } N_{o}=\text { Activity present at the end of the sampling period } \\
& \mathrm{N}=\text { Activity at some time interval, after end of sampling } \\
& \mathrm{t}=\text { Time interval } \mathrm{N}_{\mathrm{o}} \text { to } \mathrm{N} \\
& \lambda=\frac{.693}{t_{\frac{1}{2}}} \\
& t_{\frac{1}{2}}=\text { Half-life of isotope } \\
& \left.C=\frac{A \lambda}{f} \frac{1}{(1-e-\lambda t}\right) \\
& \text { Where: } C \text { Concentration per unit volume } \\
& \text { A = Activity of filter media at end of sampling period } \\
& \text { ( } N_{0} \text { from previous equation) } \\
& \mathrm{f}=\text { Sampling rate ( } \mathrm{M}^{3} / \text { minute } \text { ) } \\
& t=\text { Time samp1ing was taken } \\
& \lambda=\frac{.693}{t_{\frac{1}{2}}} \\
& t_{\frac{1}{2}}=\text { Half life of isotope or controlling parent }
\end{aligned}
$$

## APPENDIX I (cont'd.)

## RADON DETERMINATION (cont'd.)

III. Example Calculations - Room E-15
$N_{0}=\frac{498 \mathrm{dpm}}{e^{-\frac{.693 \times 104}{36}}}=3687 \mathrm{dpm}$


APPENDIX 2A

## AIR SAMPLE DATA

LOCATION: MUSEUM OF SCIENCE AND INDUSTRY E-2
LOCATION: MUSEUM OF SCIENCE AND INDUSTRY

APPENDIX 2C
LOCATION: MUSEUM OF SCIENCE AND INDUSTRY E-15
LOCATION: MUSEUM OF SCIENCE AND INDUSTRY E-18

APPENDIX 2E


APPENDIX 2F


LOCATION: MUSEUM OF SCIENCE AND INDUSTRY
LOCATION: MUSEUM OF SCIENCE AND INDUSTRY - SOuth Hall (2nd Balcony West) SAMPLE COLLECTION DATE: 4/15/77

APPENDIX 2J

## AIR SAMPLE DATA



## APPENDIX 3

## NORMAL URANIUM CALCULATIONS

Radioactive half lives of ${ }^{234} \mathrm{U},{ }^{235} \mathrm{U}$ and ${ }^{238} \mathrm{U}$ as well as the percent abundance for each isotope were obtained as current best values from the "Table of Isotopes - 6th Edition" by C. M. Lederer, J. M. Hollander and I. Perlman, 1967. The following values used are:

| ${ }^{234} \mathrm{U}$ half-1ife | $2.47 \times 10^{5}$ years |
| :--- | :--- |
| ${ }^{235} \mathrm{U}$ half-1ife | $7.1 \times 10^{8}$ years |
| ${ }^{238} \mathrm{U}$ half-1ife | $4.51 \times 10^{9}$ years |
| ${ }^{234} \mathrm{U}$ percent abundance | 0.0057 |
| ${ }^{235} \mathrm{U}$ percent abundance | 0.7196 |
| ${ }^{238} \mathrm{U}$ percent abundance | 99.2760 |

It should be noted that the abundance totals $100.0013 \%$. Since it cannot be determined which isotope(s) are in error, the calculations are made with the . $0013 \%$ error not accounted for.

```
Avagadro's Number Used - 6.025 x 1023
```

$S A=\lambda N$
$\mathrm{SA}^{234} \mathrm{U}=\frac{.693 \times 6.025 \times 10^{23}}{2.47 \times 10^{5} \times 5.256 \times 10^{5} \times 2.34 \times 10^{2}}=1.374 \times 10^{10} \mathrm{dpm} / \mathrm{gram}$
$=1.374 \times 10^{4} \mathrm{dpm} / \mu \mathrm{gram} \times 5.70 \times 10^{-5}=.783 \mathrm{dpm} / \mu \mathrm{gram}$ of normal uranium

$$
\begin{aligned}
\text { SA }^{235} \mathrm{U} & =\frac{.693 \times 6.025 \times 10^{23}}{7.1 \times 10^{8} \times 5.256 \times 10^{5} \times 2.35 \times 10^{2}}=4.76 \times 10^{6} \mathrm{dpm} / \mathrm{gram} \\
& =4.76 \mathrm{dpm} / \mu \mathrm{gram} \times 7.196 \times 10^{-3}=.034 \mathrm{dpm} / \mu \mathrm{gram} \text { of normal uranium }
\end{aligned}
$$

## APPENDIX 3 (cont'd.)

## NORMAL URANIUM CALCULATIONS

$$
\begin{aligned}
\mathrm{SA}^{238} \mathrm{U} & =\frac{.693 \times 6.025 \times 10^{23}}{4.51 \times 10^{9} \times 5.256 \times 10^{5} \times 2.38 \times 10^{2}}=7.4 \times 10^{5} \mathrm{dpm} / \mathrm{gram} \\
& =.74 \mathrm{dpm} / \text { ugram } \times 9.9276 \times 10^{-1}=.735 \mathrm{dpm} / \text { ugram of normal uranium }
\end{aligned}
$$

therefore, 1 ggram of normal uranium contains

$$
\begin{aligned}
& .783 \mathrm{dpm}{ }^{234} \mathrm{U}+.034 \mathrm{dpm} 235 \mathrm{U}+.735 \mathrm{dpm} 238 \\
= & \frac{1.552 \mathrm{dpm} / \mu g r a m}{2.22 \mathrm{dpm} / \mathrm{pCi}}=1.552 \mathrm{dpm} / \mu \mathrm{gram} \\
= & .6991 \mathrm{pCi} / \mu g r a m \text { normal uranium }
\end{aligned}
$$

Conversion of $\mu \mathrm{g} / \mathrm{gm}$ to $\mathrm{pCi} / \mathrm{g}$
Example Calculation - EP-1A

$$
3.5 \pm 0.4 \frac{\mu g r a m}{g r a m} \times \frac{0.6991 \mathrm{pCi}}{\mu \mathrm{gram}}=2.4 \pm 0.3 \mathrm{pCi} / \mathrm{gram}
$$

